

3-D Plasma Equilibrium Reconstruction at the HSX Stellarator



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Motivation

- Investigate MHD equilibrium properties via plasma reconstruction using the V3FIT code [1], experimental data and modeled signal responses [2]
- determine finite beta effects and in tokamaks to measure effects of non-axisymmetric coils
- Optimized magnetic diagnostic upgrade necessary for improved
- Necessary to find most sensitive diagnostics for the plasma equilibrium currents to be measured
- Understand the limitations of plasma reconstruction performance

Plasma Reconstruction

describe plasma size, plasma current and plasma pressure profile

$$p(s) = PRESSCALE \cdot \left[\frac{1}{1 - K} \left[\left(1 + \left(\frac{s}{AM(2)} \right)^{AM(3)} \right) \right] \right]$$

$$K = \left[1 + \left(\frac{s}{AM(3)} \right)^{-1} \right]$$

 $I(s) = CURTOR \cdot \left(\frac{2}{\pi}\arctan\left(\frac{AC(2)s^{AC(1)}}{1-s}\right)\right)$

- plasma equilibrium [3]
- minimizing

$$\chi^{2} = \sum_{i} \left(\frac{S_{i}^{\text{model}}(h_{j}, k_{j}) - S_{i}^{\text{measure}}}{\sigma_{i}} \right)^{2}$$

- $h = \{PRESSCALE, AM(2), AM(3)\}$
- By means of plasma reconstruction. current was proven at HSX [4]

Free parameters in plasma current and pressure profile

PRESSCALE → pressure

Increase in $AC_2 \rightarrow shift$

Increase in $AC_3 \rightarrow shift$

 Φ =0 (ellipse shape)

to right and peaking

Set of two external previously installed magnetic diagnostic coils

(measuring toroidal, poloidal, and radial field components)

 Φ =1/2 Field Period

to left and peaking

at magnetic axis

 $AM_3 \rightarrow steepness$

 $AM_2 \rightarrow width$

- Three dimensional plasma reconstruction is necessary in stellarators to
- HSX offers test bed for plasma reconstruction and benchmarking algorithms
- reconstruction by solution reduction and increased signal-to-noise ratio

$$p(s) = PRESSCALE \cdot \left[\frac{1}{1 - K} \left[\left(1 + \left(\frac{s}{AM(2)} \right) \right) \right] \right]$$

$$K = \left(1 + \left(\frac{s}{AM(2)} \right)^{AM(3)} \right)^{-1}$$

- VMEC is used to compute the
- Plasma reconstruction is based on

$$^{2} = \sum_{i} \left(\frac{S_{i}^{\text{model}}(h_{j}, k_{j}) - S_{i}^{\text{measure}}}{\sigma_{i}} \right)^{2}$$
tion, between measured signals.

deviation between measured signals S^{measure}, and predicted values S^{model}, by adjusting the model parameters

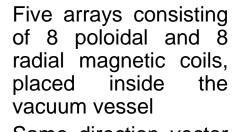
- $k = \{CURTOR, AC(2), AC(3)\}$ [1]
- Two external and one internal magnetic coil array (at two different toroidal positions) in continuous use
- helical structure of Pfirsch-Schlüter

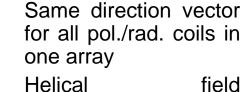
Diagnostic Setup

- Three different methods applied to find the location for a new set of magnetic diagnostics to improve plasma equilibrium reconstructions on HSX
- 1. Single Value Decomposition Study [2]
 - → Diagnostic set for maximizing detection of different magnetic field pattern
- 2. Signal Effectiveness Study (SEFFS)
 - → Most effective diagnostic for minimizing the uncertainty in the reconstructed
- 3. Single parameter dependency study (SPDS)
 - → Diagnostics sensitive to change in specific reconstruction parameter

Diagnostic Construction

structure





- included due to low signal-noise ratio
- Fixed coils between allows of the magnetic
- Flat design to lie 1cm outside the LCFS
- Ceramic used for coil body

Pair of Helmholtz coils

Metal shielding only

frequencies

for each direction

above 20 kHz

Suppression

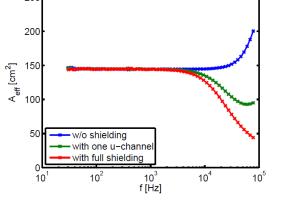
self resonance

used to calibrate coils

- Stainless steel used as shielding for the coils
- Each array can be separately calibrated, replaced, and upgraded with more coils

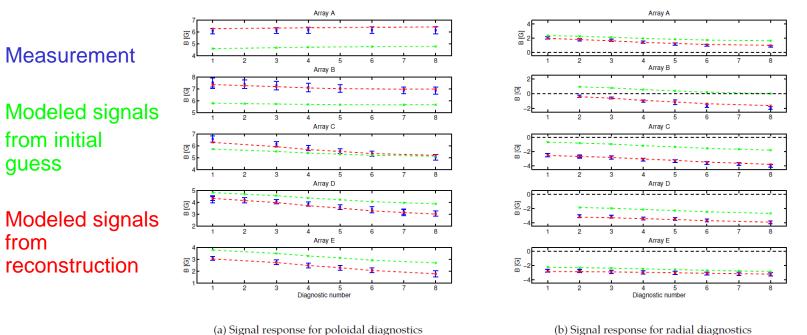
Diagnostic calibration

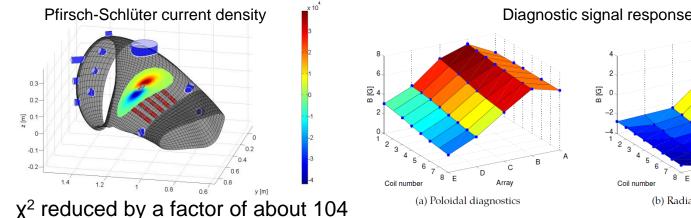




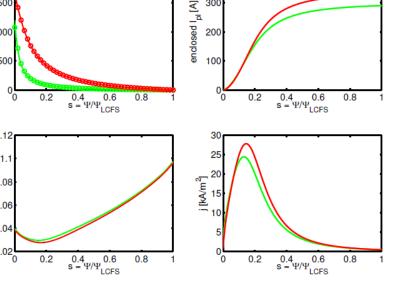
Reconstruction Results

- discharges on right (average over 11
- Diagnostics used in reconstruction process
 - 74 Bdot coils (new array)
- Diagnostics used for initial guess and reconstruction result comparison:
 - Rogowski coil (total plasma current)
 - Diamagnetic loop (stored energy)
 - Thomson Scattering (pressure profile)
- Bootstrap current dominates plasma ≤ 300 current evolution
- Time of reconstruction 0.848-0.849s





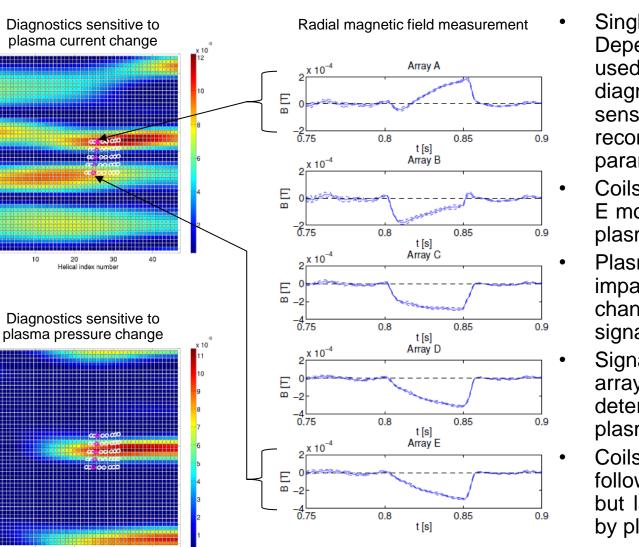
- 13 % deviation in reconstructed Rogowski coil measurement
- energy Reconstructed plasma over plasma pressure) factor of 2.4 too high compared to diamagnetic loop and Thomson Scattering measurement
- Use of exclusively radial or poloidal diagnostics yield reconstruction results
- Different initial conditions, fits for the pressure profile, reconstruction of earlier time steps do not show a reduced discrepancy → more investigation necessary



(b) Radial diagnostics

External Rogowski coi

Diagnostic Sensitivity



- Parameter Dependence Study diagnostic location sensitive to specific reconstruction parameters
- Coils in array A and E mostly sensitive to plasma current
- leads to change in sign of signal in A vs. E
- Signal of coils in C mostly array determined plasma pressure
- Coils in array B follow stored energy but later determined by plasma current

Summary

- Plasma equilibrium reconstruction at HSX is based on the determination of seven free parameters describing plasma current profile, pressure profile and plasma size
- Internal diagnostic set of 80 coils built based on 3 different positioning concepts
- Helmholtz coils used to calibrate each diagnostic coil for each direction
- Possibility to design diagnostic coil set sensitive to specific MHD properties experimentally proven
- QHS plasma reconstructed solely with new array to benchmark existing diagnostics
- Relatively small discrepancy in reconstructed total plasma current
- Relatively large discrepancy in plasma pressure profile is still under investigation

References

- 1) J.D. Hanson et al. *Nucl. Fusion* 49 (2009)
- 2) S.P. Hirshman et al. *Phys. Plas.* 11 (2004)
- 3) S.P. Hirshman and J.C. Whitson. *Phys. Fluids*, 26 (1983)
- 4) J.C. Schmitt Dissertation UW-Madison (2011)
- 5) N. Pomphrey et al Phys. Plasmas 14 (2007)

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